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Status of complete neutron-induced displacement damage cross section data

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Introduction



- Nuclear analysis support for design and safety of nuclear and accelerator facilities demand accurate and validated nuclear data.
- Specific interest in fusion technology applications, i.e. DT 14-MeV neutrons and d-Li neutron sources (IFMIF-DONES)
- Radiation damage due to irradiation with neutrons poses limitations on material performances and systems' lifetimes.
- European effort on provision of recommended nuclear data libraries, in particular nuclear response files – displacement damage.
- Libraries should cover a wide range of elements and energies.
 - Joint Evaluated Fission and Fusion (JEFF) nuclear data library. <u>JEFF Nuclear Data Library - NEA (oecd-nea.org)</u>



Methodology, Codes, Data, Assumptions

- Damage models and parameters
- Production of displacement cross sections
- Uncertainty propagation
- Processing to nuclear data libraries

Damage Models and Parameters

NRT

M.J. Norgett et al. (1975)

$$N_{NRT}(T) = \frac{0.8}{2E_d} \frac{T}{1 + k_L g(T/E_L)}$$
$$g(\epsilon) = \alpha_1 \epsilon^{1/6} + \alpha_2 \epsilon^{3/4} + \alpha_3, \quad \alpha_4 = E_d$$
$$\alpha_1 = 3.4008, \quad \alpha_2 = 0.40244, \quad \alpha_3 = 1.0$$

Fusion (E < 14 MeV) 1.2 Fission (E_< 10 MeV) 1.0 N/^{dd} 0.8 N = 0.6 toller'99(MD) Original MD used by OECD fit OECD fit % uncertainty used in calculation-Stoller'12(MD) 0.4 Ed(=40eV) ₽*Ed(= Sivak'03(MD) 0.2 Ortiz'17(BCA.MD SRIM(BCA.MD) *0.8 S.P. Simakov et 0.0 0.03 0.1 100 1000 10 Damage Energy of Recoil, keV

athermal recombination corrected dpa(arcdpa)K. Nordlund et al. (2015)

$$N_{arcdpa}(T) = \left(\frac{1 - c_{arc}}{(2E_d/0.8)^{b_{arc}}} T^{b_{arc}} + c_{arc}\right) N_{NRT}(T)$$

$$\alpha_5 = b_{arc}, \ \alpha_6 = c_{arc}$$

al. (2018)

$$10^{-1}$$
 10^{-1} 10



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Damage Models and Parameters





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10²

101

10-4

10-3

10-2

10-1

T_{PKA} (MeV)



Production of Displacement Cross Sections

- Data processing via NJOY2016, and other tools (e.g. PREPRO 2021)
- 78 Elements (Be...Bi) from JEFF-4T2.2 (also: JENDL-5, TENDL-2021, ENDF/B-VIII.0)
 - Unphysical jumps in a series of crosssections (Mg, Al, Si, P, S, Ar, K, Ca, Sc, Ti, Cr, Ni, Zn, Ga, Ge, As, Se, Pd, Sm, Tb, Er, Tm, Yb)
 - Extension to 200 MeV, where necessary





Production of Displacement Cross Sections

1200

1000

800

600

400

200

0 L

Displacement cross-section (b)



JEFF-4T1	all data: 22 %,	3 ≤ Z ≤ 83: 13.5 %
TENDL-21	all data: 0.8 %,	3 ≤ Z ≤ 83: 0.5 %
JENDL-5	all data: 27 %,	3 ≤ Z ≤ 83: 20 %
ENDF/B-VIII	all data: 92 %,	3 ≤ Z ≤ 83: 91 %

Bridging and smoothing



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Uncertainty Propagation



MC Method (D.L. Smith, 2004)

- Choice of basic set of model parameters
- Estimation of standard deviations
- MC sampling of input data sets
- Calculation of output data sets
- Statistical evaluation

$$p_{k} = \{p_{01} \pm \Delta p_{k1,...,p_{0L}} \pm \Delta p_{kL}\}$$
$$V_{i,j} = (1/K) \sum_{k=1,K}^{K} (\sigma_{ki} - \sigma_{0i}) (\sigma_{kj} - \sigma_{0j})$$

 $\Delta \sigma_{d,ii} = \sqrt{V_{ii}}$

Uncertainty Propagation, Assumptions



- Atomic displacement models $\Delta E_d / E_d = 20\%$
 - **NRT**: $\Delta \alpha_1 / \alpha_1 = \Delta \alpha_2 / \alpha_2 = \Delta \alpha_3 / \alpha_3 = 15\%$

Arc-dpa: $\Delta b_{arc}/b_{arc} = \Delta c_{arc}/c_{arc} = 20\%$

- Nuclear data uncertainties
 - Covariance matrices from current JEFF/TENDL (light targets: previous TENDL)
 - Random sampling from cross section covariances



Uncertainty Propagation, Sampling



- Covariance matrices for displacement cross-sections from parameter uncertainties
 - Atomic displacement models
 - Covariance matrices from current JEFF/TENDL (light targets: previous TENDL)



Uncertainties and Correlations





Summary and data



NRT and arc-dpa displacement cross sections with covariance matrices

