



Impact of nuclear data on the design of fluoride cooled reactors

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Fluoride-cooled high-temperature reactors feature solid fuel and molten salt coolant



 Solid fuel: fuel is in the form of pebbles, plates, or blocks containing coated-particle fuel (TRISO) developed for high-temperature gas-cooled reactors (HTGRs) with failure temperatures >1650°C



Coolant: high-temperature, low-pressure liquid-salt coolant (⁷Li₂BeF₄) with freezing point of 460°C and boiling point >1400°C (transparent)





Data of importance for FHRs

- Thermal scattering data
 - Graphite
 - Flibe
- Cross sections of ¹⁹F, ⁹Be, ^{6/7}Li





The major source of uncertainty in flibe is ⁷Li capture cross section

| Reaction | Uncertainty, pcm | |
|--------------------------|------------------|--|
| ⁷ Li capture | 1,240 | |
| ²³⁵ U nu | 379 | |
| ²³⁸ U capture | 214 | |
| ¹⁹ F capture | 172 | |
| ²³⁵ U capture | 157 | |
| ²³⁵ U fission | 138 | |
| ¹² C capture | 138 | |
| ¹² C elastic | 121 | |
| Total | 1,380 | |

FHRs feature relatively small coolant density reactivity coefficients

- Coolant density (temperature) reactivity feedback in FHRs is a fine balance between flibe absorption and moderation
 - Positive feedback from reduced absorption
 - Negative feedback from reduced moderation (spectrum hardening)
- Coolant temperature feedbacks can only be achieved if flibe has a significant contribution to moderation



Coolant temperature reactivity feedback (pcm/K) as a function of carbon-to-heavy metal ratio



Uncertainty in the coolant density feedback is also dominated by ⁷Li capture





⁹Be(n,alpha) cross section determines ⁶Li concentration

- ⁶Li cannot be completely eliminated as it produced by (n,alpha) reactions on ⁹Be
- Concentration of ⁶Li reaches an equilibrium that depends on cross sections

$$N_{Li-6} = \frac{\sigma_{Be-9}^{\alpha}}{\sigma_{Li-6}^{abs}} N_{Be-9}$$

 1 ppm of ⁶Li is worth ~175 pcm reactivity



⁹Be(n,alpha) cross section





FHR and fluoride molten salt reactors share similar data needs

Uncertainty from nuclear data for the effective multiplication factor of the Molten Salt Reactor Experiment

| Reaction | Uncertainty, pcm | | |
|--------------------------|------------------|--|--|
| ²³⁵ U nu | 373 | | |
| ¹² C elastic | 263 | | |
| ²³⁸ U capture | 257 | | |
| ⁷ Li capture | 197 | | |
| ²³⁵ U capture | 171 | | |
| ¹⁹ F elastic | 143 | | |
| ²³⁵ U fission | 120 | | |

Chloride molten salt reactors are impacted by the large uncertainties in ³⁵Cl cross sections







There are large discrepancies between data libraries for ³⁵Cl(n,p) cross section



The main discrepancies are in the fast energy range



ENDF/B-VII.1 includes detailed resonances between ~0.1 MeV and ~1 MeV

This is more realistic than the smooth trend adopted so far by other libraries

The cross section in this region is 2-3 orders of magnitude smaller when using ENDF/B-VII.1 vs. ENDF/B-VII.0

In fast systems most neutrons are in this energy region



The difference between ENDF/B-VII.0 and ENDF/B-VII.1 is 5200 pcm!



| Quantity | ENDF/B-VII.0 | ENDF/B-VII.1 | Difference |
|---|-------------------|-------------------|------------|
| k∞ | 1.06789 ± 0.00023 | 1.11989 ± 0.00021 | +0.05200 |
| (n,p) effective cross section, b | 0.01947 | 0.01109 | -0.00839 |
| (n, γ) effective cross section, b | 0.00245 | 0.00246 | +0.00001 |

Switching from ENDF/B-VII.0 to ENDF/B-VII.1 is equivalent to increase ³⁷Cl enrichment to about 50%







New measurements were made at UC Berkeley







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