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Nuclear Data for MYRRHA



NURA

With its **NURA** project, SCK CEN significantly increases its contribution to the fight against cancer. By pooling its knowledge and expertise in terms of radiopharmaceuticals, NURA contributes to the development of the next-generation radiopharmaceuticals. More specifically, NURA performs game-changing research into radiopharmaceuticals for treating different types of cancer in cooperation with clinical and industrial partners.



MYRRHA

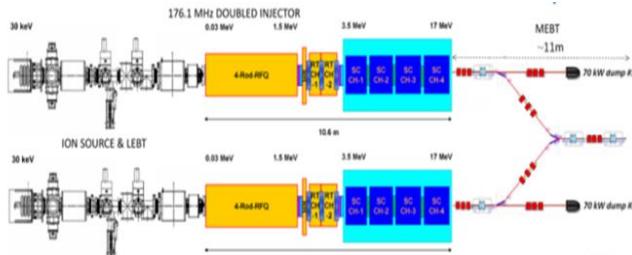
SCK CEN works actively on the design and construction of a new multi-purpose research plant: MYRRHA, which stands for *Multi-purpose HYbrid Research Reactor for High-tech Applications*. MYRRHA is a versatile research infrastructure but above all unique. It is the world's first research reactor driven by a particle accelerator.

RECUMO

With the public-public partnership RECUMO, SCK CEN and the *National Institute for Radio Elements* (IRE) reach out to one another. SCK CEN will decontaminate the current and future highly radioactive residues and thus reduce the stock. In this way, RECUMO contributes to the security of supply of medical radio-isotopes, which are indispensable in the fight against cancer.

MYRRHA

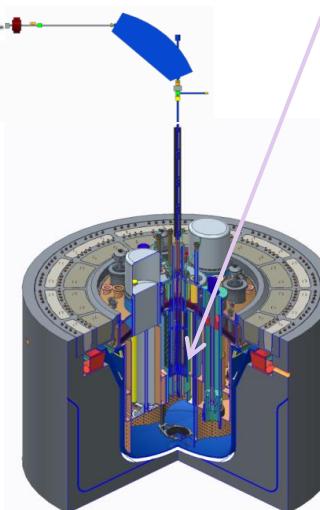
- MYRRHA – An Accelerator Driven System
 - Demonstrate the ADS concept at pre-industrial scale
 - Can operate in critical and sub-critical modes
 - Demonstrate transmutation
 - Fast neutron source



Accelerator	
Particles	protons
Beam energy	600 MeV
Beam current	2.4 to 4 mA

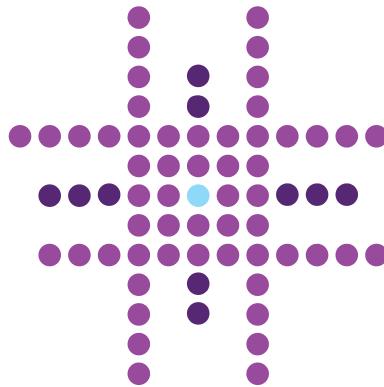
Reactor	
Power	65 to 100 MW _{th}
k_{eff}	0.95
Spectrum	fast
Coolant	LBE

Target	
Main reaction	spallation
Output	2·10 ¹⁷ n/s
Material	LBE (coolant)

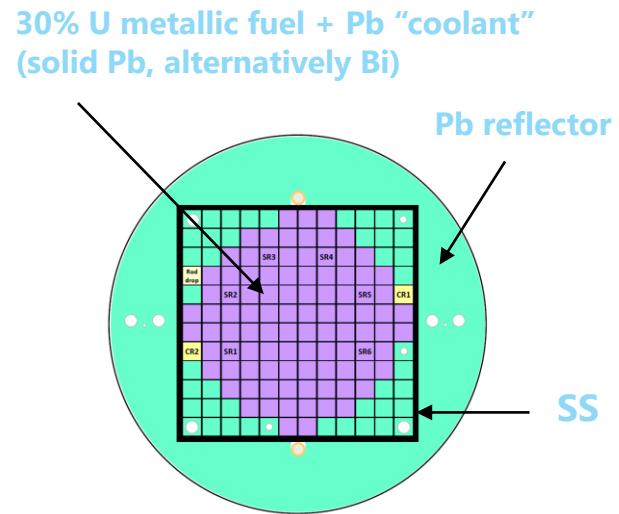
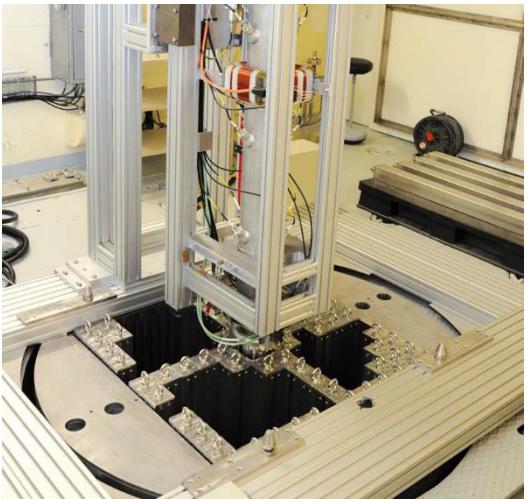


MYRRHA design

- Codes
 - Core
 - MCNP6.2
 - ALEPH2
 - Accelerator
 - MCNP6.2
 - ALEPH2
 - PHITS
- Nuclear data
 - JEFF-3.1.2, JEFF-3.2, JEFF-3.3 & JEFF-4T0
 - ENDF/B-VII.0, ENDF/B-VII.1 & ENDF/B-VIII.0
 - JENDL-4.0 & JENDL-5beta
 - TENDL-2014, TENDL-2015, TENDL-2017 & TENDL-2019

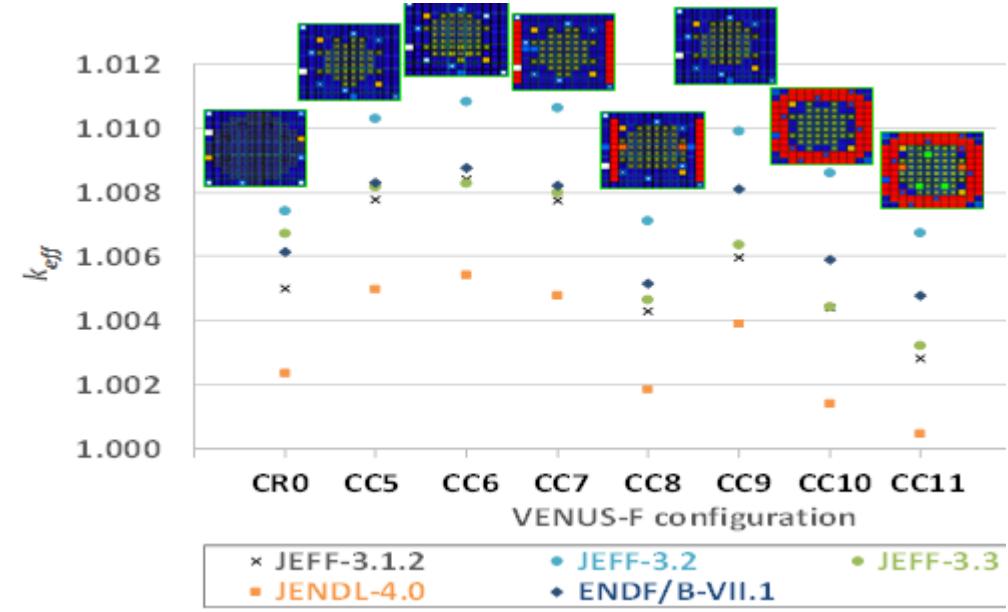


Nuclear Data Validation: VENUS-F



Core	#FAs	FA composition	Reflector	In-Pile Section
CR0	97	9 U+16 Pb	Pb	-
CC5	41	13 U+8 Pb+4 Al ₂ O ₃	Pb	-
CC6	41	13 U+8 Pb+4 Al ₂ O ₃	Pb	-
CC7	41	13 U+8 Pb+4 Al ₂ O ₃	Pb+C	-
CC8	47	13 U+8 Pb+4 Al ₂ O ₃	Pb+C	thermal spectrum
CC9	41	13 U+8 Bi+4 Al ₂ O ₃	Pb	-
CC10	41	13 U+Pb+8 Bi+4 Al ₂ O ₃	Pb+C	-
CC10b	47	13 U+Pb+8 Bi+4 Al ₂ O ₃	Pb+C	thermal spectrum
CC11	50	13 U+Pb+8 Bi+4 Al ₂ O ₃	Pb+C	thermal and fast spectrum

Source: A. Kochetkov and P. Baeten



Besides criticality, we have:

- Kinetic parameters
- CR curve
- Spectral indices
- Axial and radial traverses
- Pb-Bi void
- Fuel Doppler

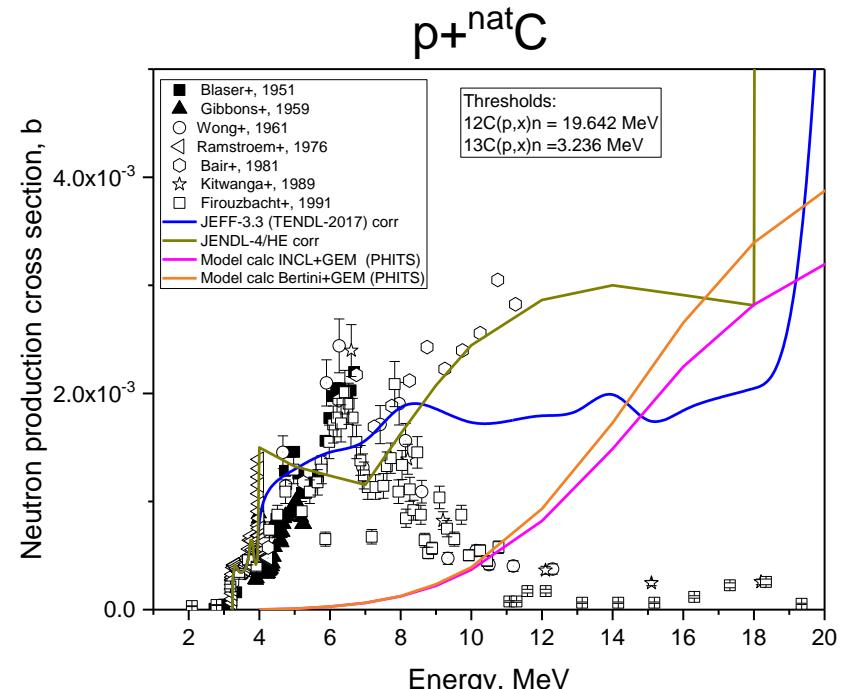
Extensive database for ND validation!

Nuclear Data Needs

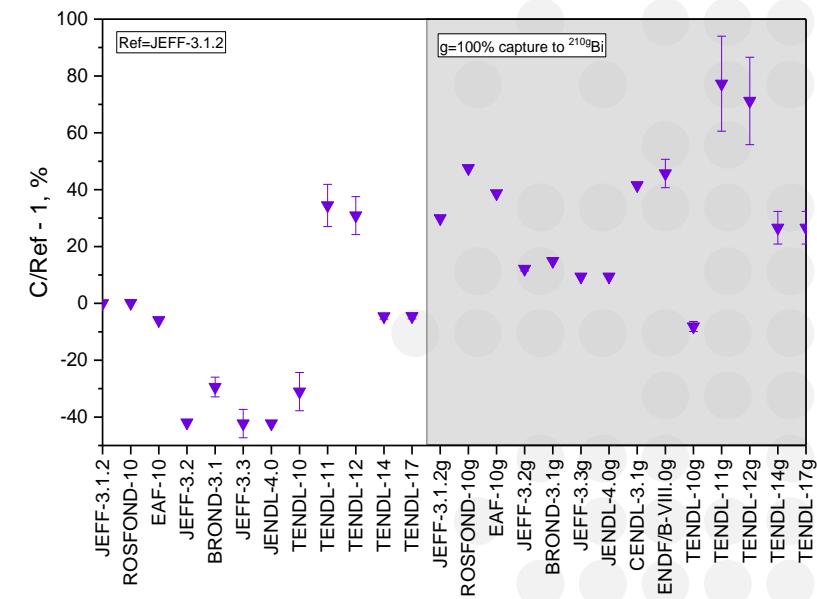
Nuclear data needs in JEFF-3.3 for MYRRHA:

- Adoption of JENDL-4.0 evaluation for ^{204}Pb or re-evaluation in the RRR and URR
- New evaluation $^{57}\text{Fe}(\text{n},\text{inel})$ including missing resonances
- Re-evaluation $^{10}\text{B}(\text{n},\text{inel})$ uncertainty
- Covariance evaluation for $^{209}\text{Bi}(\text{n},\text{n})$ and $^{209}\text{Bi}(\text{n},\gamma)$
- Covariance evaluation for v_{T} , v_{p} , v_{d} ^{240}Pu & v_{d} $^{235,238}\text{U}$ and $^{239,242}\text{Pu}$
- Reduction of uncertainty $^{240}\text{Pu}(\text{n},\text{f})$
- Reduction of uncertainty $^{54,57}\text{Fe}(\text{n},\text{n})$
- Reduction of uncertainty $^{208}\text{Pb}(\text{n},\text{n})$
- Reduction of uncertainty $^{238}\text{U}(\text{n},\text{inel})$

Criticality



Shielding



Radioactive source term
and waste management

Source: JEFFDOC-1994, JEFFDOC-1956 and P. Baeten

Benchmarks



- Criticality
 - VENUS-F - MYRRHA mockup
 - Different configurations for nuclear data validation
- Shielding
 - Double-differential neutron yields experiments
 - Neutron transmission experiments
- Nuclear data adjustment
 - Simple configuration
 - Highly sensitive to a single nuclide and reaction channel

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